

# INVESTIGATION OF INCINERATION RATE OF $^{241}\text{AM}$ , $^{243}\text{AM}$ AND $^{237}\text{NP}$ SPALLATION TARGETS USING A PROTON ACCELERATOR DRIVEN SYSTEM USING MCNPX CODE

Z. Gholamzadeh,  
S.A.H. Feghhi,  
C. Tenreiro,  
M. Rezazadeh

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# Nuclear waste

Mutation  
And  
cancer

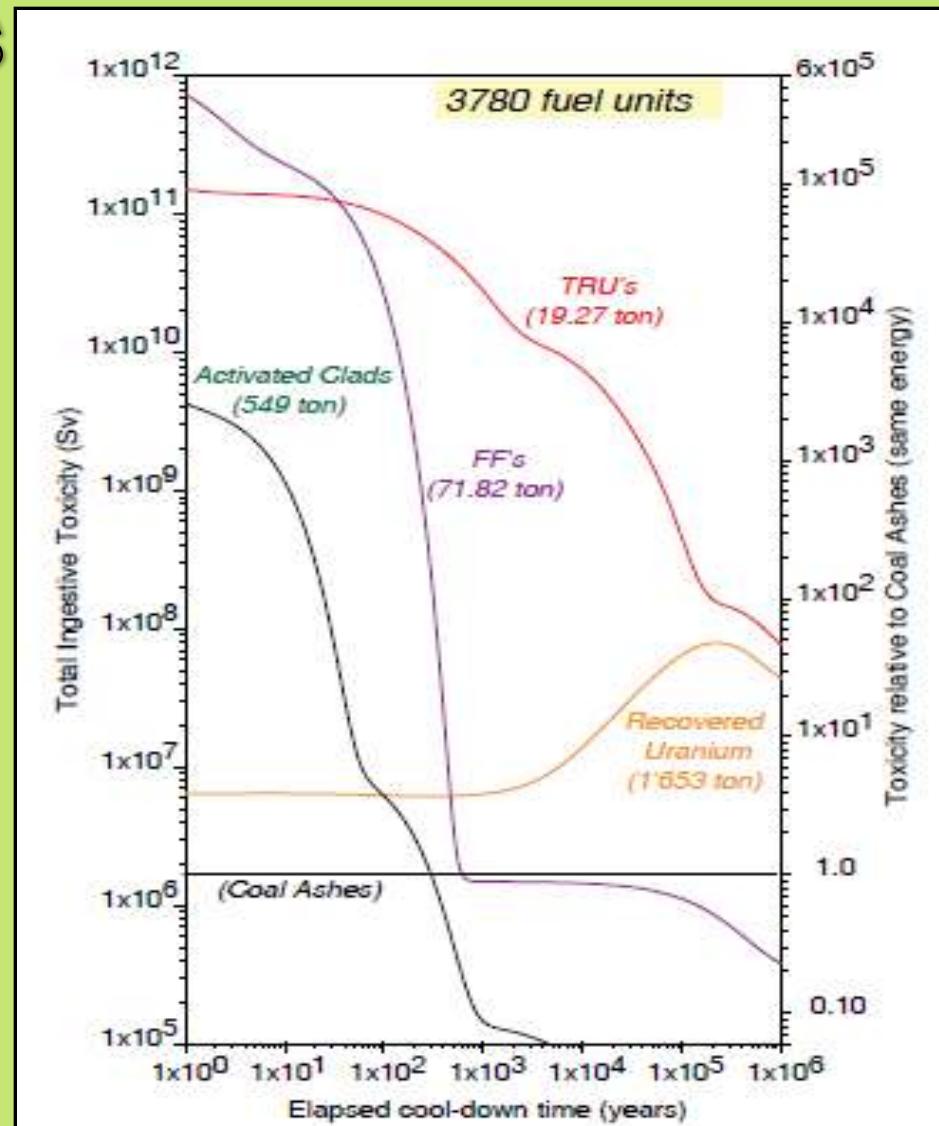
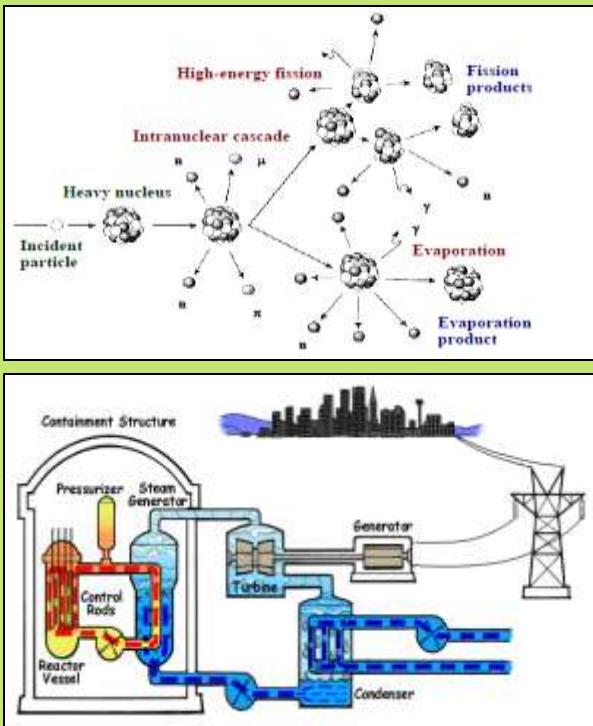


□ In the world are 442 operating nuclear reactors  
400,000 ton of waste produce annually: about 12,000 ton  
is high-level waste (**HLW**).

- 96% of this is U
- 1% of this are actinides (TRU transuranics)
- 3% other fission products

# The most radiotoxic elements

- Transuranic elements
- Fission products

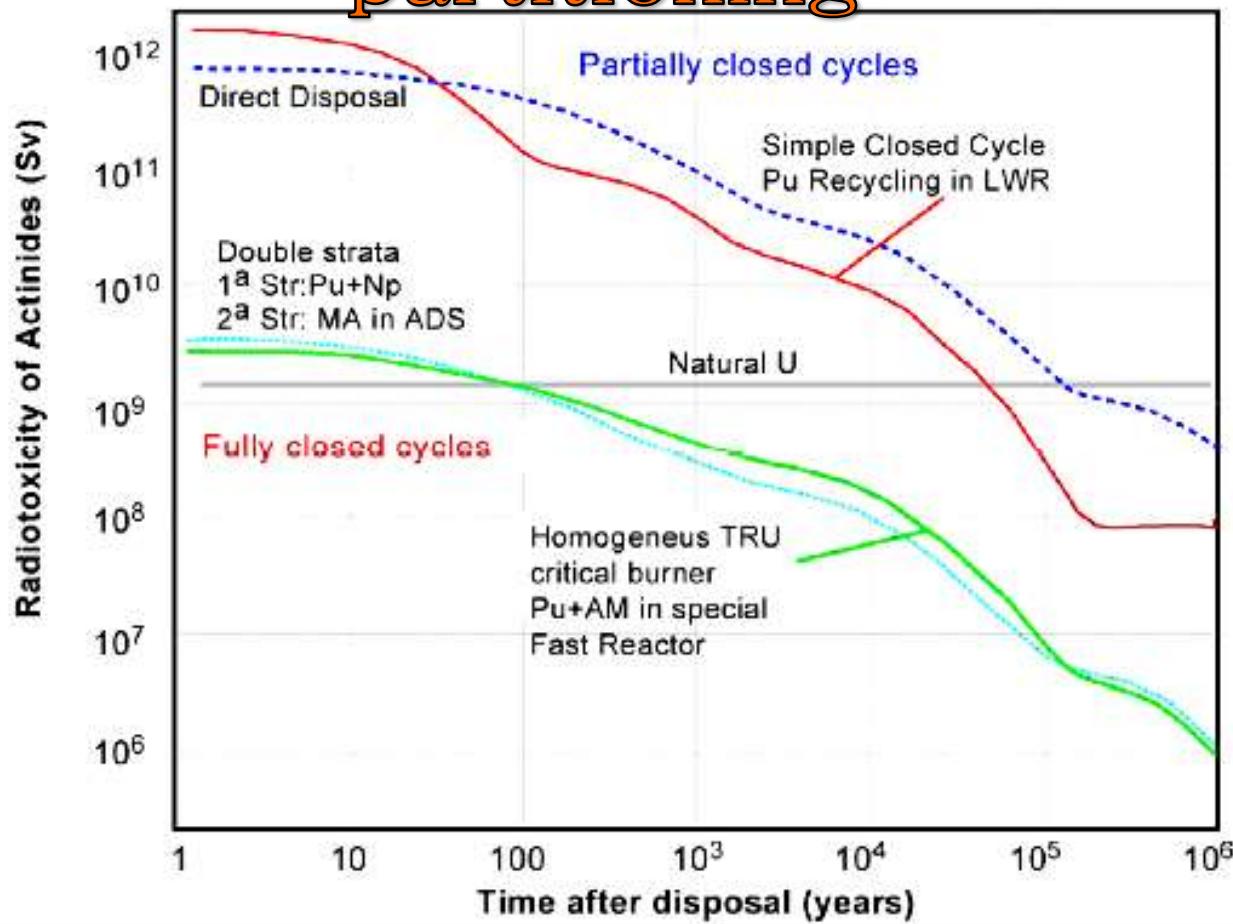


# Nuclear waste disposal

- ❖ Geological disposal of the UK's radioactive waste is circa **16.44 \$ billion**
- ❖ The value is **15 \$ billion** in U.S.



# Transmutation and partitioning



# SNS cost

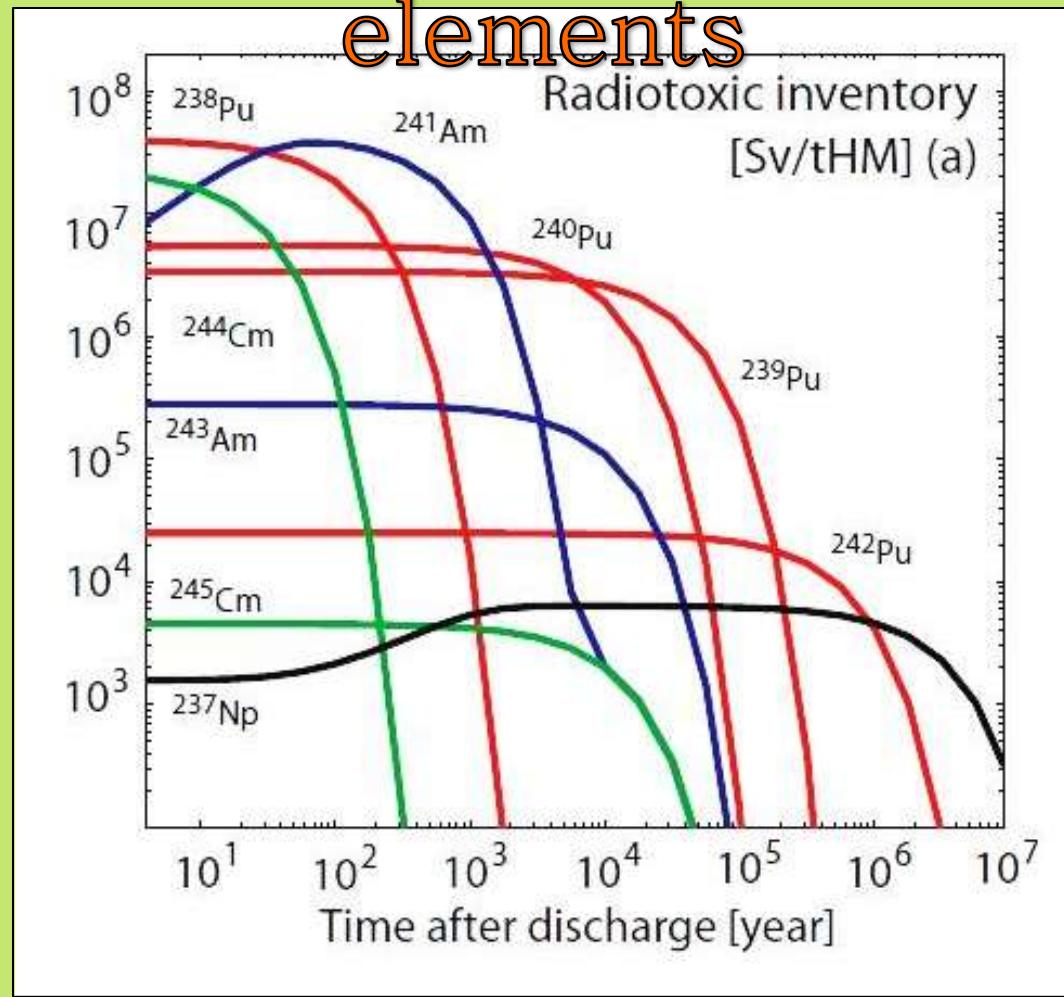
The total project cost of SNS was  
**1.4B\$**

Proton beam energy on target	<b>1.0</b>	<b>GeV</b>	SC linac output energy	<b>1.0</b>	<b>GeV</b>
Proton beam current on target	<b>1.4</b>	<b>mA</b>			
Power on target	<b>1.4</b>	<b>MW</b>	HEBT length	<b>170</b>	<b>m</b>
Pulse repetition rate	60	Hz			
Beam macropulse duty factor	6.0	%	Accumulator ring circ.	<b>248</b>	<b>m</b>
Ave. current in macro-pulse	26	mA	Ring fill time	<b>1.0</b>	<b>m</b>
H <sup>+</sup> peak current front end >	<b>38</b>	<b>mA</b>	Ring beam extraction gap	<b>250</b>	<b>ns</b>
Chopper beam-on duty factor	68	%	RTBT length	<b>150</b>	<b>m</b>
RFQ output energy	2.5	MeV			
FE + Linac length	335	m	Protons per pulse on target	<b><math>1.5 \times 10^{14}</math></b>	
DTL output energy	87	MeV	Proton pulse width on target	<b>695</b>	<b>ns</b>
CCL output energy	185	MeV	Target material	<b>Hg</b>	



# The most radiotoxic elements

- ❖ Pu
- ❖ Am
- ❖ Cm
- ❖ Np



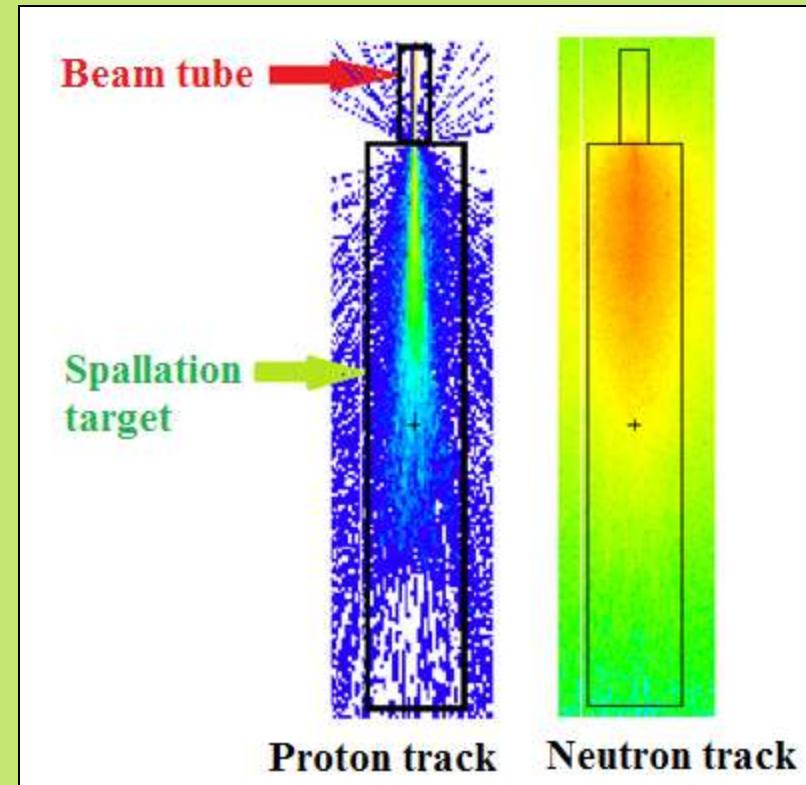
# Objectives of this simulation work

## Work proposal:

- ▶ Proton energies: 0.4, 0.6, 0.8 and 1 GeV
- ▶ Spallation targets:  $^{241}\text{Am}$ ,  $^{243}\text{Am}$  and  $^{237}\text{Np}$
- ▶ Cylindrical target dimension  $10 \times 60$  cm
- ▶ MCNPX 2.6.0 Monte Carlo-based code

## Calculations:

- ▶ Escaped neutron yield from the spallation targets
- ▶ Neutron absorption rate
- ▶ Leaked neutron spectra from the spallation targets
- ▶ Heat deposition inside the spallation targets
- ▶ Radionuclide production inside the spallation targets
- ▶ Effective multiplication factor, delayed neutron fraction and effective delayed neutron fraction

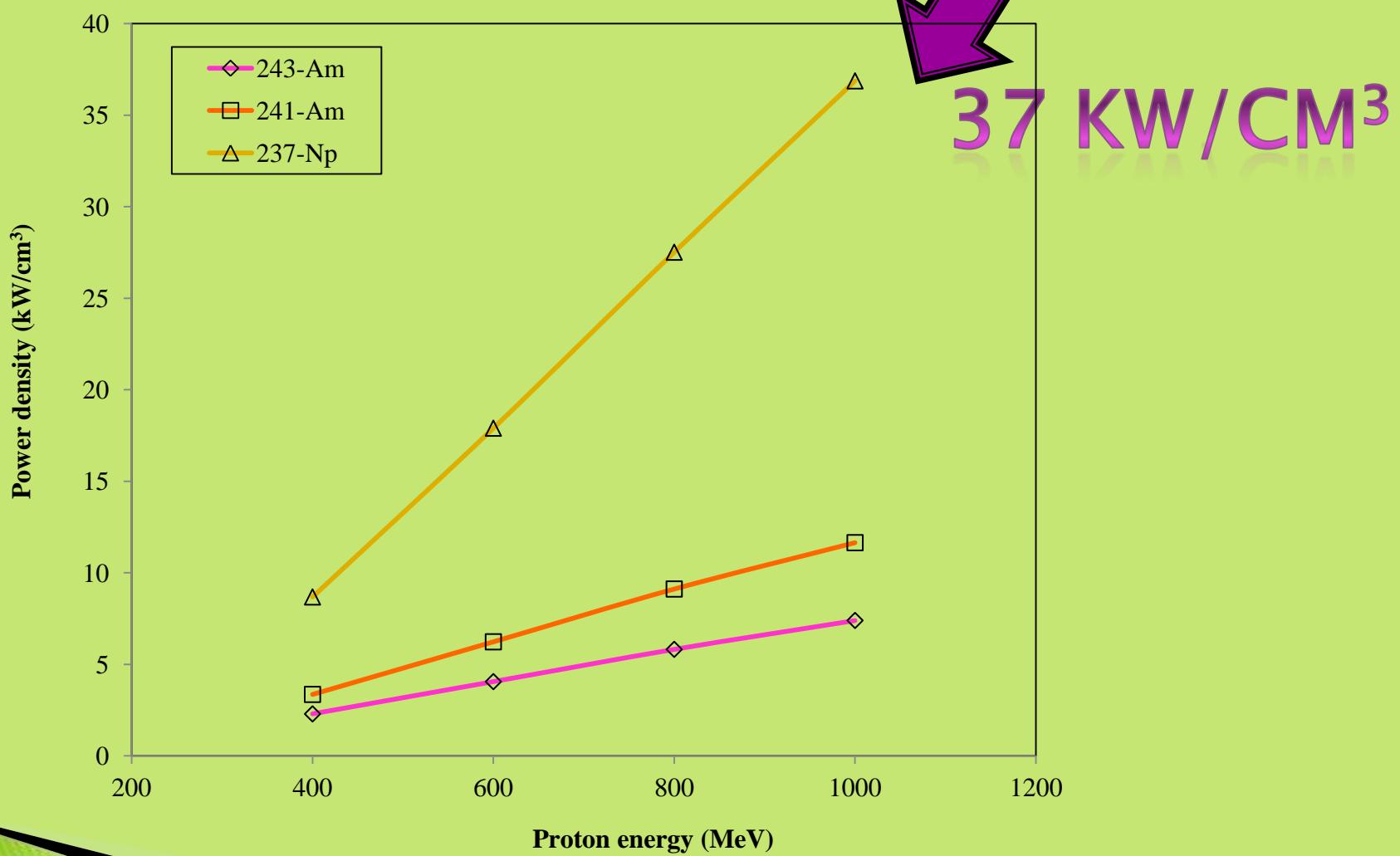








# Deposited heat





# SNS

## neutronic

Comparison of different physical and neutronic properties of spallation targets

Target specifications	$^{237}\text{Np}$	$^{241}\text{Am}$	$^{243}\text{Am}$
Density ( $\frac{g}{cm^3}$ )	20.25	13.67	13.67
Melting point ( $^\circ\text{C}$ )	644	1176	1176
Boiling point ( $^\circ\text{C}$ )	4000	2011	2011
Termal conductivity ( $\frac{W}{mK}$ )	6	10	10
$\beta$ (pcm)	194	214	190
$\beta_{eff}$ (pcm)	75	231	182
Neutron per fission ( $v$ )	3.00	3.67	3.87
$k_{eff}$	0.86213	0.76451	0.64974

# SNS

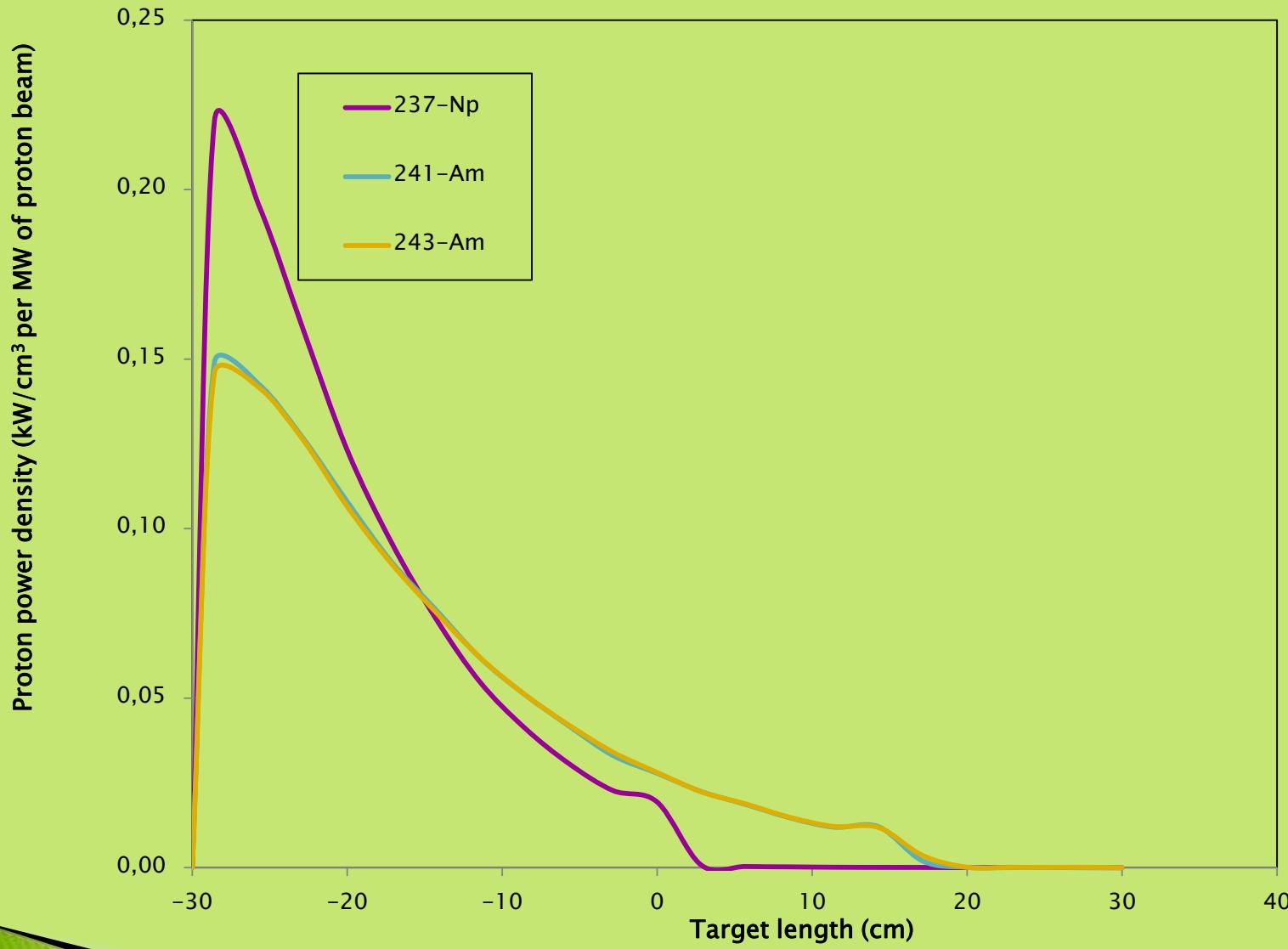
## neutronic

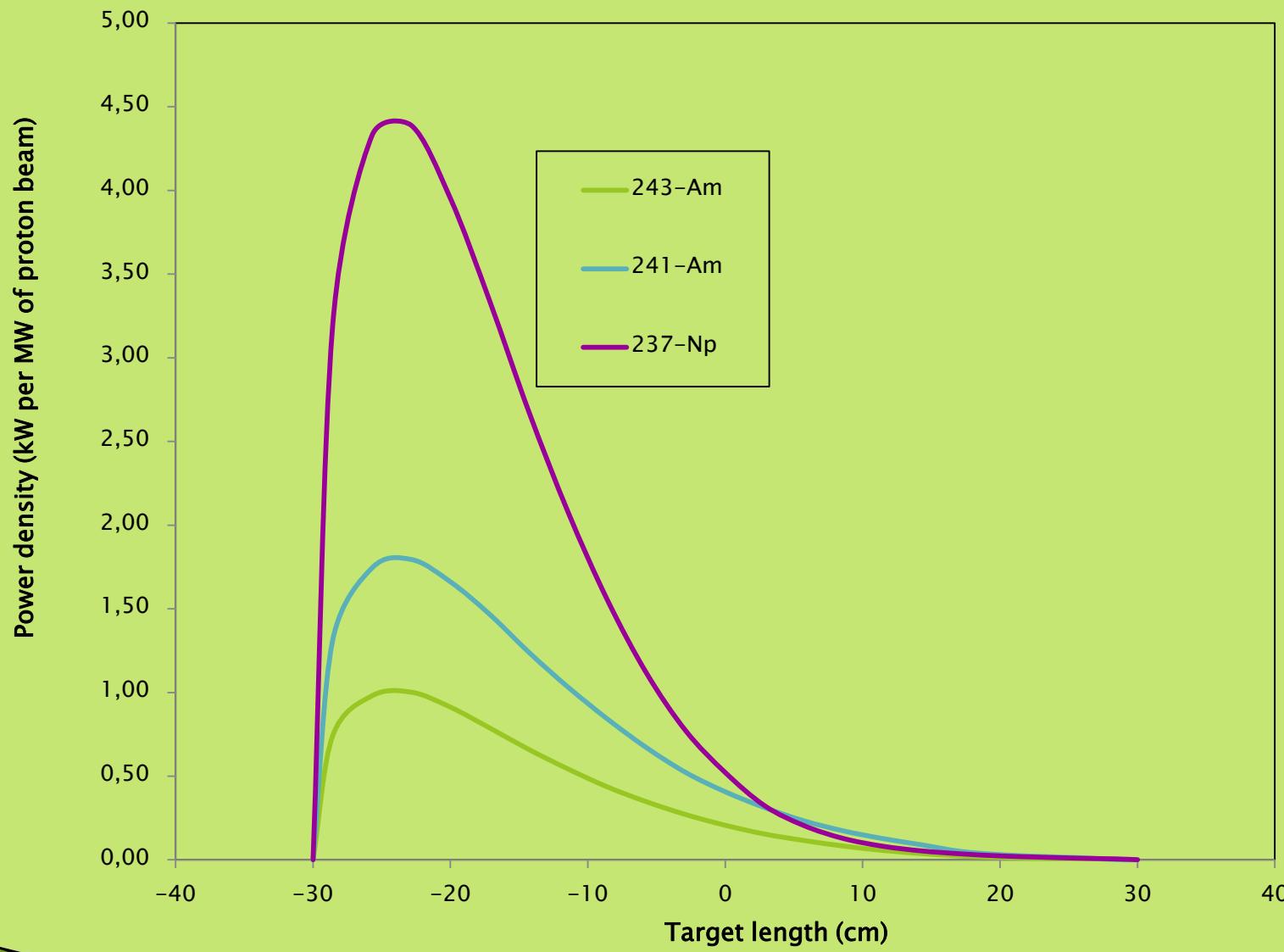
Comparison of different neutronic properties of spallation targets

Quantity	Energy (GeV)			
	0.4	0.6	0.8	1
<b><math>^{237}\text{Np}</math></b>				
(n,G) (#/s)	2.72E+16	5.77E+16	9.39E+16	1.23E+17
(n,f) (#/s)	8.74E+16	1.82E+17	2.97E+17	3.89E+17
Transmutation rate (kg/y)	1.423651	2.980534	4.850051	6.362937
Gas production (atom/y)	2.40E+08	3.47E+08	4.92E+08	6.48E+08
<b><math>^{241}\text{Am}</math></b>				
(n,G) (#/s)	1.15E+16	2.35E+16	3.59E+16	4.71E+16
(n,f) (#/s)	5.35E+16	1.08E+17	1.65E+17	2.16E+17
Transmutation rate (kg/y)	0.820617	1.661986	2.536969	3.326655
Gas production (atom/y)	2.39E+08	3.35E+08	4.59E+08	6.01E+08
<b><math>^{243}\text{Am}</math></b>				
(n,G) (#/s)	8.41E+15	1.68E+16	2.49E+16	3.25E+16
(n,f) (#/s)	2.73E+16	5.41E+16	8.09E+16	1.05E+17
Transmutation rate (kg/y)	0.451024	0.894867	1.335081	1.736473
Gas production (atom/y)	2.31E+08	3.24E+08	4.43E+08	5.77E+08



*Thank you for your  
attention*





**Table 1-** Energy range of neutron spectra leaked from the spallation targets.

Neutron energy	$^{237}\text{Np}$ (%)	$^{241}\text{Am}$ (%)	$^{243}\text{Am}$ (%)
<b>1e-04 MeV&lt;En&lt;1 MeV</b>	4.47E+01	3.76E+01	4.43E+01
<b>1 MeV&lt;En&lt;20 MeV</b>	5.39E+01	6.06E+01	5.27E+01
<b>20 MeV&lt;En&lt;1000 MeV</b>	1.33E+00	1.88E+00	2.98E+00

Table 1  
The composition of MA in the spent fuel of a light water reactor

Isotopes	Mass (kg/year) per unit PWR <sup>a</sup>
<sup>237</sup> Np	15.1
<sup>238</sup> Pu	16.1
<sup>239</sup> Pu	205
<sup>240</sup> Pu	120
<sup>241</sup> Pu	72.7
<sup>242</sup> Pu	41.6
<sup>241</sup> Am	6
<sup>242m</sup> Am	0.00793
<sup>243</sup> Am	21.8
<sup>244</sup> Cm	15.6
<sup>245</sup> Cm	1.74

33 MWd/kg, 32.5% thermal efficiency, 150 days after discharge.

Nuclear Chemical Engineering (Manson et al., 1981, p. 370, Table 8.5).

<sup>a</sup> Pressurised-water reactor, fuel with plutonium recycle, 1000-Mwe reactor, 80% capacity factor.

reactors can become viable (Boczar et al., 2002a,b; Critoph, 1976; Jagannathan et al., 2001; Loewen et al., 2001). Other studies have investigated the utilization of LWR spent fuel in CANDU reactors mixed with thorium ( $\text{ThO}_2$ ) (Şahin et al., 2004a,b).

A series of studies have been conducted on the Direct Use of spent Pressurized water reactor (PWR) fuel In CANDU reactors (DUPIC) by Korean researchers in a wide spectrum, which covered a wide spectrum of physical and technological aspects, including fuel management scenario and fuel performance (Choi et al., 1997).

In the present work, the neutronic analysis of a CANDU reactor with a mixed fuel made of thorium and MA is presented. The utilization of MA in combination with thorium has two main purposes. Firstly, it will enable the incineration of MA and so the reduction of the long living nuclear waste material. Furthermore, it will allow the

Table 1 Some important records of neutron targets so far been used at various spallation neutron facilities

Target	Proton Energy, MeV	Thermal Cycles	Total Protons, mA hrs	Peak Temperature, °C	Time-average beam current, μA	Relative Total Number of Fissions to U #5
ISIS U #1	780	—	92.4	—	30	0.31
ISIS U #2	800	40000	53.1	120	40	0.18
ISIS U #3	800	10389	174.9	130	65	0.59
ISIS U #4	800	4147	138.8	150	75	0.47
ISIS U #5	800	5074	295.6	165	90	1.00
ISIS U #6	800	2628	126.1	180	110	0.43
ISIS U #7	800	1805	107.2	215	125	0.36
ISIS U #8		Not Used			—	—
ISIS U #9	800	815	113.2		150	0.381
IPNS Depleted #1	450	89600	240.0	225	10 - 14	0.39
IPNS Enriched #1	450	28000	128.8	175	14 - 16	1.07
KENS U #1	500	~ 40000	~ 50	~ 120	~ 5	~ 0.1
ISIS Ta #1	800	73378	1751.6		170	3.80**
ISIS Ta #2*	800	21138	618.1		170	1.34**

\* Still in use