

Quasi-infinite uranium and thorium targets irradiated by deuterons /protons with relativistic energies

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This presentation analyzes the reactions rates (n,γ), (n,n'), (n,2n), (n,f) and the fission neutron multiplicity in a "Big" uranium or thorium targets.

Defines a quasi-infinite target (medium) as a function
of beam energy

• Simulations and experiments

Wasil'kov-Goldanski set-up and experiment

simulation of future experiments with Buran set-up

Neutron production in targets (238U, depU and 232Th) by irradiation with high energy protons and deuterons

1. Spallation reactions



- 2. Fragmentation of the uranium/thorium core.
- neutron multiplicity and neutron spectra
- **3. Neutrons induced fission (n,f).**
- neutron multiplicity, neutron spectra and fragments
- 4. Charged particles and gammas induced fission (p,f), (d,f), (π ,f) and (γ ,f).
- neutron multiplicity, neutron spectra and fragments
- 5. Reactions (n,2n), (n,3n) and (n,4n).
- 6. Radiative capture and inelastic interactions (n,γ) and (n,n').

The aim of ADS

1. Energy production.

Neutron induced fission of ²³⁸U and ²³²Th.

♦ production of ²³⁹Pu by radiative capture:

$${}^{238}_{92}\mathrm{U} + {}^{1}_{0}\mathrm{n} \longrightarrow {}^{239}_{92}\mathrm{U} \xrightarrow{\beta^{-}}_{23.5 \mathrm{\ min}} {}^{239}_{93}\mathrm{Np} \xrightarrow{\beta^{-}}_{2.3565 \mathrm{\ d}} {}^{239}_{94}\mathrm{Pu}$$

♦ production of ²³³U by radiative capture:

$$n + {}^{232}_{90}Th \rightarrow {}^{233}_{90}Th {}^{\beta^-}_{\rightarrow} {}^{233}_{91}Pa {}^{\beta^-}_{\rightarrow} {}^{233}_{92}U$$

2. Transmutation of nuclear waste and possibility to use it as fuel for ADS.

MCNPX calculation of "BIG" ²³⁸U, ^{depl.}U and ²³²Th targets

1. MCNPX limitations for ^{238,235}U and ²³²Th.

Incident neutrons energy for the reactions (n,f), (n,xn)
^{235,238,233}U and neutron multiplicity are defined up to E_n=20MeV.
Charged particles and photons induced fission are not included.

Incident neutrons energy for the reactions (n,f), (n,xn)
²³²Th and neutron multiplicity are defined up to E_n=60MeV.
Charged particles and photons induced fission are not included.

Simulations and nuclear data

Target: Cylinder from 238 U or 232 Th. Size L=1 – 4m, R=0.4 – 1.2 m

and Buran set-up

MCNPX 2.7e transport code.

Nuclear models: CEM03, INCL4+ABLA, QGSM

ENDF and TENDL data tables.

For the reaction rate calculations were used:

1. FM card

$$C\int_{E1}^{E2} \Phi \Box E \Box \sigma \Box E \Box dE$$

- 2. Output file
- 3. Convolution method $N = \sum \Phi \Box E \Box \sigma \Box E \Box dE$
- 4. Fission energy deposition

Cross sections (n,f), (n, γ) and (n,n') and multiplicity for ²³⁵U and ²³⁸U



Multiplicity for of U isotopes

 Average fission neutron multiplicity as a function of incident neutron energy is known quite well for nuclei ²³⁵U, ²³⁸U up to 200 MeV



Alexander laptev, LANL

Gamma induced fission in ^{235,238}U



Deuteron and protons induced fission. Cross sections for ²³⁵U and ²³⁸U



Cross sections (n,f)²³³U and multiplicity constant



Cross sections (n,f)²³²**Th and multiplicity constant**



Vassil'kov- Goldanski experiment. Buran experimental se



Target assembly of the experiment. Dimensions of uranium target 56x56x64cm, 3.5 tons. The thickness of the lead shield is 10cm.

Buran set up cross sections. It is composed of two materials: depleted uranium, 21 tons (central part D=18x80cm and the rest part) and steel container.

MCNPX calculations + est. impact of high energy neutrons, a=0.25 (5) Vasil'kov-Goldanski set-up with Buran set-up

E _d [GeV]	Vasil'kov-Goldanski set-up, ^{nat} U	Buran set-up, ^{nat} U	
N _{totmcnpx}	69	80	
N _{tot est}	114	127	
Nescape	15	2	
(n,f) _{MCNPX}	10	11	
(n , γ) _{MCNPX}	32	42	
$N\gamma_{totest}$	66	80	
Nf tot est	22.3/(22exp.+-10%)	24	
BPG _{est}	8	8.9	

Quasi infinite-targets

all high energy particles (charged particles and neutrons) deliver their energy in the volume. The reactions rate of (n,f), capture, (n,xn) and neutron leakage are approximately constant with the increase of the target volume

Quasi targets

The high energy neutrons induced reactions turn approximately constant, but the radiation capture and neutron leakage are still increasing and decreasing respectively with the increase of the target volume Calculation of (n,f), $(n,\gamma)^{depl}U$ reactions for Buran set up. The average multiplicity for incident neutrons with energies up to $E_n < 20$ MeV is $<\mu>=3.2$ neutrons, for $E_n > 20$ MeV is <m>=8.5 neutrons and for charged particles induced fission is $<\mu>=10$ neutrons. The leakage of the neutrons is about 5-10 %

1	E _d [GeV]	1	2	4	6	12			
2		MCNP	X calculation	S					
3	N _{tot}	129	288	567	823	1536			
4	$N_{\gamma tot}$	73	164	325	473	883			
5	N_{ftot} , E _n <20MeV	15.3	34.5	68	98	184			
6	N_{ftot} , total $\mathbf{E_n}$ spectrum	18.9	42.7	84	122	227			
7	$N_f(\mathbf{p}+\mathbf{d}+\pi+\gamma,\mathbf{f})^{235,238}\mathbf{U}$	1.2	2	2.3	5	9.7			
8	BPG _{MCNPX}	6.15	6.26	5.7	5.54	5.14			
9	MCNPX calculations + estimated impact of high energy neutrons								
10	N _{tot est}	200	450	884	1297	2386			
11	$N_{\gamma tot est}$	123	276	545	803	1472			
12	$N_{f tot est}$	37	83	163	216	439			
13	BPG	9.4	10.3	9.8	8.9	8.8			

CONCLUSIONS

- Quasi-infinite target for targets of ²³⁸U and ²³²Th for E_d=8GeV are: uranium cylinder D=120cm, L=80cm thorium cylinder D=200 cm, L=160.cm
- 2. Vasil'kov-Goldanski set up is a not quasi-infinite targets for Ep=660MeV
- 3. Buran set-up can by assumed to be a quasi-infinite target for deuteron beam with energy up to Ed=8GeV (MCNPX calculations)
- 4. Up to now there have been no experiments with quasi-infinite spallation targets made of (Pb, W, Bi) or fission nuclides, irradiated by relativistic particles.