

Cement-polymer composites containing PANI/B₄C: Neutron shielding performance by Monte Carlo simulation

Setenay Tunckilic, Deniz Agehan Kahraman*, Ayse Nur Esen

Istanbul Technical University, Energy Institute, Istanbul, Türkiye

*E-mail: yalcinkayad19@itu.edu.tr

The International Seminar on Interaction of Neutrons with Nuclei (ISINN),
April 14–18, 2024, Sharm El-Sheikh, Egypt

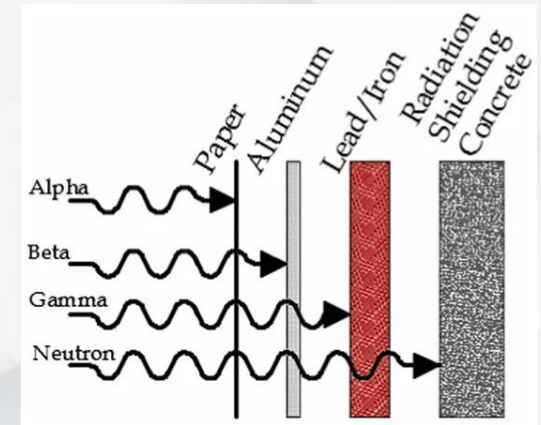
1. Purpose of the Study

- Investigating the use of cement composites containing Polyaniline (PANI) and Boron carbide (B_4C) to develop a new material for neutron shielding:

Neutron radiation requires special protection provided by shielding materials due to its high penetration ability.

Cementitious materials are common neutron shielding materials due to their cost-effective and reliable shielding performance.

Enhancing the radiation shielding capabilities of cementitious materials is essential for ensuring the safety of neutron radiation facilities.



Wang, K.; Ma, L.; Yang, C.; Bian, Z.; Zhang, D.; Cui, S.; Wang, M.; Chen, Z.; Li, X. Recent Progress in Gd-Containing Materials for Neutron Shielding Applications: A Review. *Materials* 2023, 16, 4305. <https://doi.org/10.3390/ma16124305>

Radiation Physics and Chemistry 219 (2024) 111675



Contents lists available at [ScienceDirect](https://www.sciencedirect.com)

Radiation Physics and Chemistry

journal homepage: www.elsevier.com/locate/radphyschem



Neutron and gamma-ray shielding effectiveness of novel polyaniline composites

Deniz Agehan Kahraman^{a,*}, Fatma Tuba Cogalmis^b, Ayse Nur Esen^a, Sevilay Hacıyakupoglu^a, Bahire Filiz Senkal^b

^a Istanbul Technical University, Energy Institute, Maslak, 34469, Istanbul, Türkiye

^b Istanbul Technical University, Faculty of Science and Letters, Maslak, 34469, Istanbul, Türkiye

- Novel thermal neutron shielding polyaniline composite was prepared by boric acid.
- Experiments, theoretical calculations, and Monte Carlo simulations were done.
- A detailed uncertainty evaluation of results was performed.

➤ Boron carbide (B_4C):

- Highly efficient neutron shielding material due to its high ^{10}B content.
- Its use is limited in cementitious materials because of the formation of boric acid, which reduces durability.

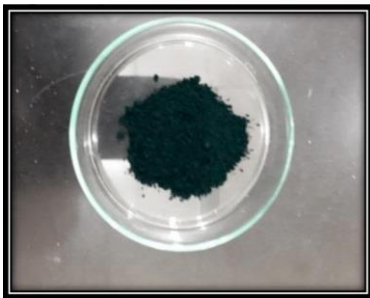


<https://www.samaterials.com/spherical-boron-carbide-powder-b4c.html>

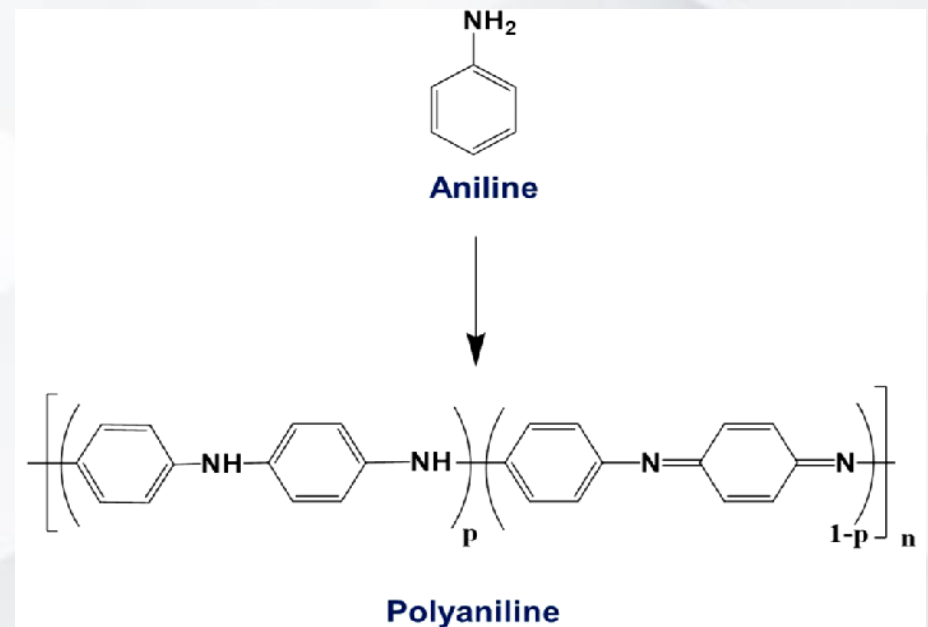
To offer an alternative solution to this problem, polymers are widely used because of their low cost, versatility, precise controllable synthesis, and availability, making them an excellent option for radiation shielding.

➤ Polyaniline (PANI):

- PANI ($C_6H_5NH_2$) is made up of a chemically flexible amino group in the polymeric chain that is bonded to any side of the phenylene ring.
- Stable in the environment
- Easy to synthesize
- Stable electrical conductivity
- Low-cost
- High-temperature resistance



Essa, Abbas & Hasan, Salma & Jerjack, Najlaa & Hassan, Salma. (2021). Optical and Structural Properties of Prepared Polyaniline - Graphene (PANI/GN) Nanocomposite.



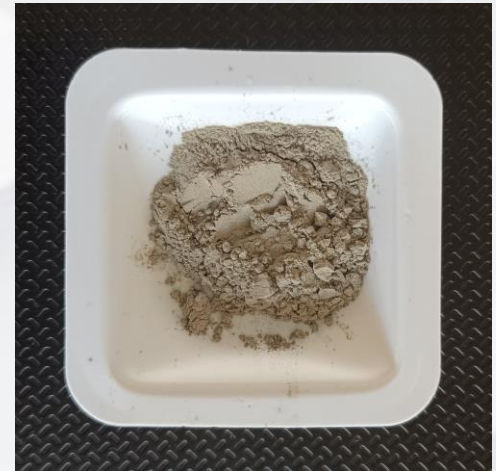
Ahmad, S., Hammad, R. & Rubab, S. Gamma Radiation-Induced Synthesis of Polyaniline-Based Nanoparticles/Nanocomposites. J. Electron. Mater. 51, 5550–5567 (2022). <https://doi.org/10.1007/s11664-022-09823-0>

➤ **Cement:**

Type: CEM I 42,5 R Portland Cement

Usage areas:

- In the production of all kinds of reinforced concrete structures such as residential buildings, bridges, foundations, viaducts, concrete road pavements, precast structural elements where early strength is important
- Ideal for use in the production of shotcrete, dry mix mortars, construction chemicals.



➤ Chemical Composition of Cement:

Symbol	Concentration
CaO	70.14
Na ₂ O	10.47
Al ₂ O ₃	6.16
Fe ₂ O ₃	3.86
SiO ²	3.43
MgO	3.43
SO ₃	1.38
K ₂ O	0.67

➤ Composition of cement composite materials

Sample code	Sample Content	wt./cement			
		Cement	DI water	PANI	B ₄ C
S1	OPC (ordinary portland cement)	100	30		
S2	OPC + 4 wt.% PANI	100	30	4	
S3	OPC + 4 wt.% B ₄ C	100	30		4
S4	OPC + 4 wt.% PANI/B ₄ C	100	30	2	2

➤ Neutron Shielding

The Beer-Lambert law was used to compute the macroscopic cross-section of the composites by Geant4 simulation.

$$I = I_0 e^{-(\Sigma \cdot x)}$$

where I_0 and I are incident and transmitted neutron flux ($\text{cm}^{-2} \cdot \text{s}^{-1}$) respectively, Σ is macroscopic cross-section (cm^{-1}), and x is material thickness (cm).

➤ Neutron Shielding

The theoretical macroscopic cross-section was calculated from microscopic cross-section (σ , cm²) and the atomic number density (N, atoms·cm⁻³) as follows:

$$\Sigma = \sigma N$$

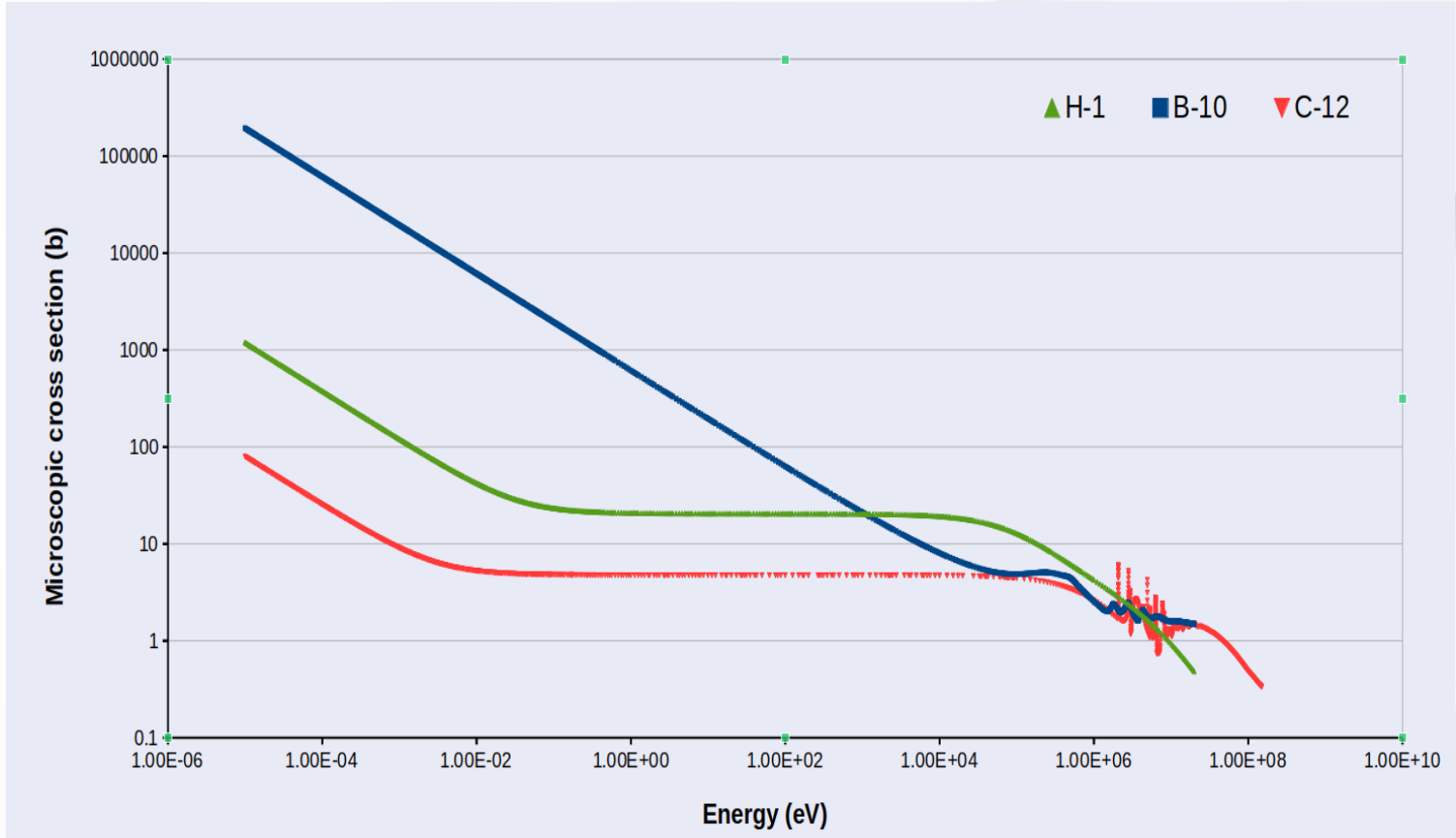
If the material is a compound instead of a simple element, the total macroscopic cross-section is the sum of the macroscopic cross-sections of the individual elements

$$\Sigma = \sum_i w_i \frac{\rho}{A_i u} \sigma_i = w_i \Sigma_i$$

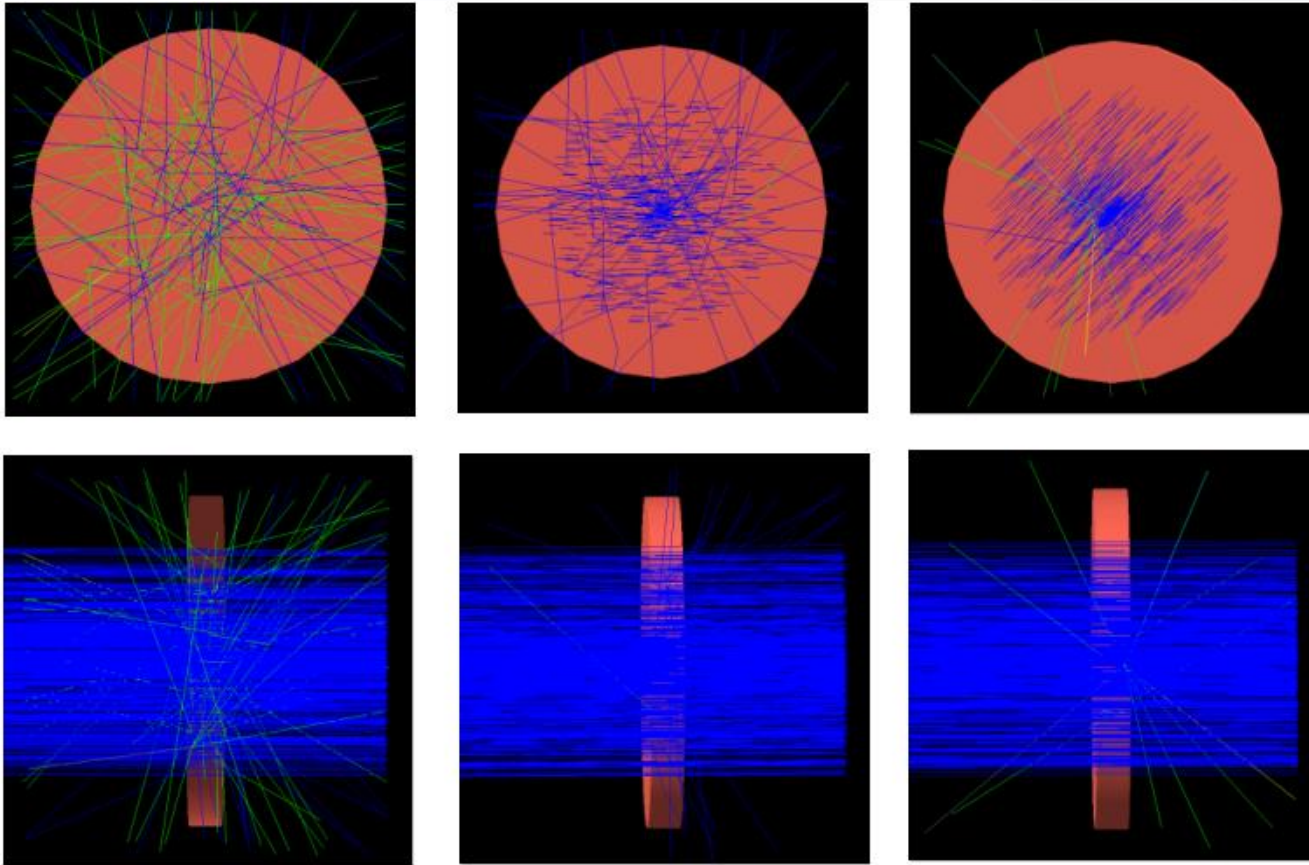
The material thicknesses required to reduce radiation intensity to 50% of their initial intensity is known as the half-value layer (HVL, cm). For neutrons, they were calculated by:

$$\text{HVL} = \ln(2) / \Sigma$$

➤ Neutron Shielding



➤ Monte Carlo Simulations



a)

b)

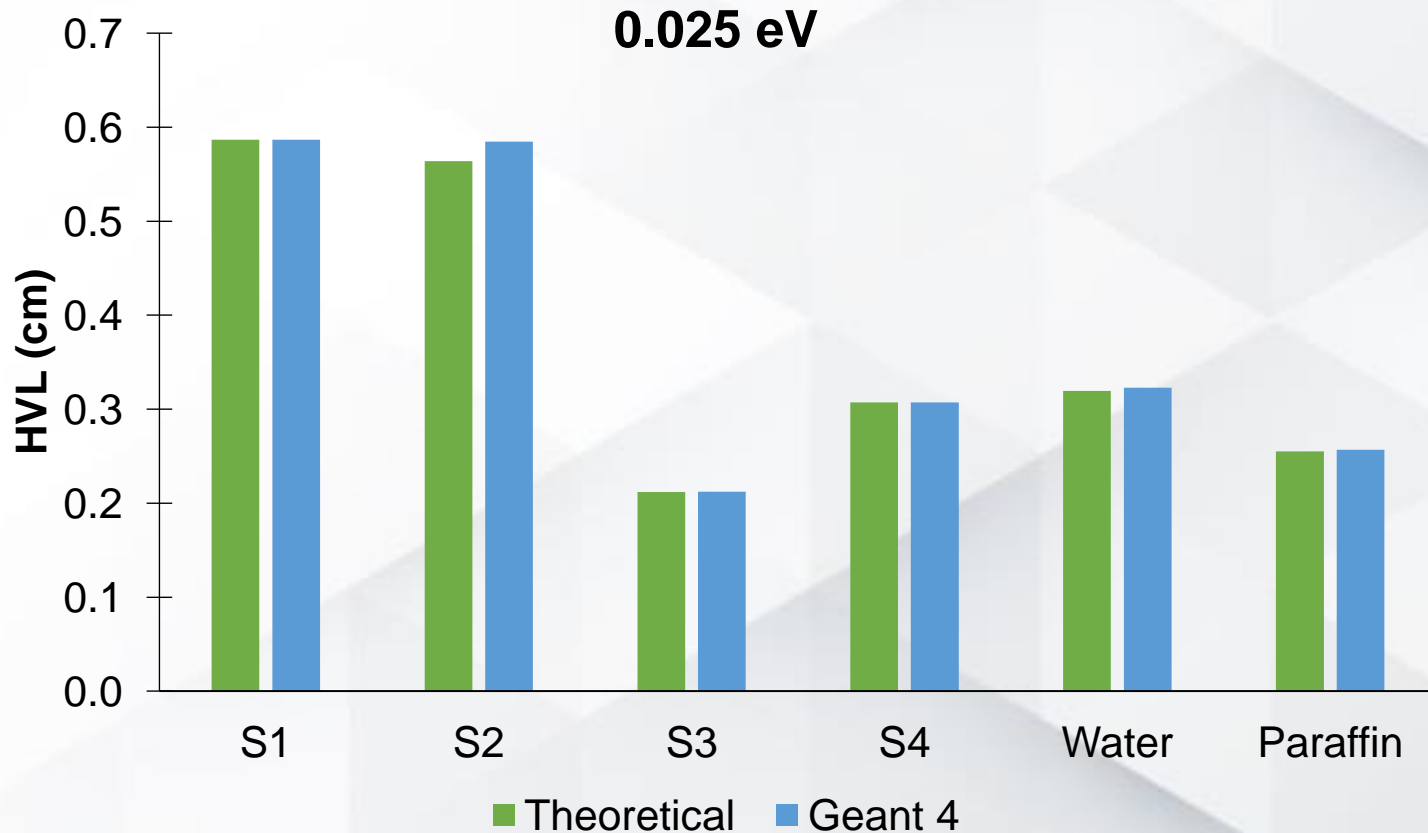
c)

Interaction of neutrons with S4 in Geant4 a) 0.025 eV b) 1 keV c) 10 MeV

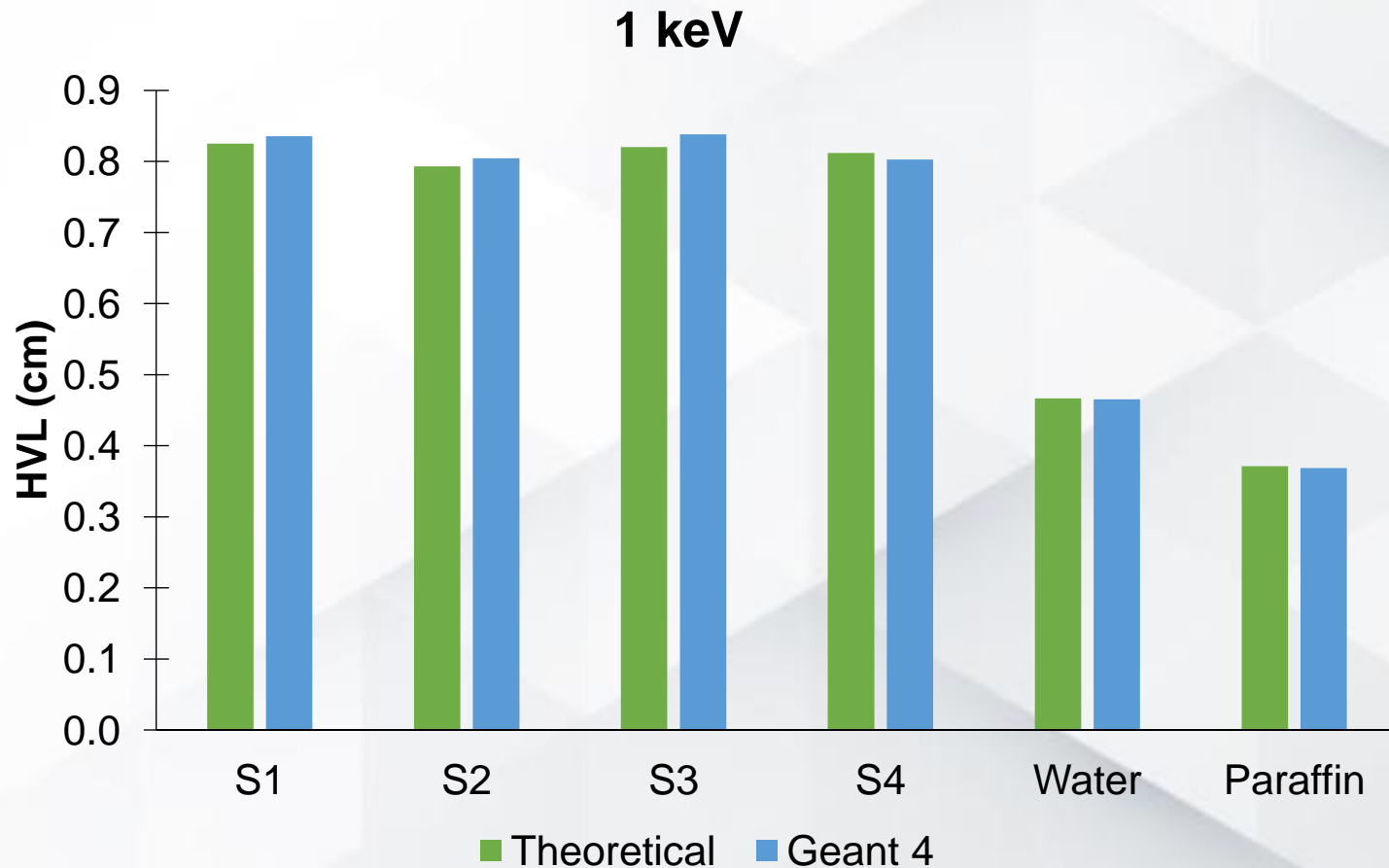
➤ **Macroscopic cross-sections (cm^{-1}) for cement-polymer composites :**

	0.025 eV		1 keV		10 MeV	
	Σ_{theo}	Σ_{Geant4}	Σ_{theo}	Σ_{Geant4}	Σ_{theo}	Σ_{Geant4}
S1	1.18	1.18	0.84	0.83	0.12	0.12
S2	1.23	1.19	0.87	0.86	0.12	0.12
S3	3.27	3.27	0.84	0.83	0.12	0.12
S4	2.26	2.26	0.85	0.86	0.12	0.12
Water	2.17	2.15	1.49	1.49	0.11	0.11
Paraffin	2.72	2.70	1.87	1.88	0.12	0.13

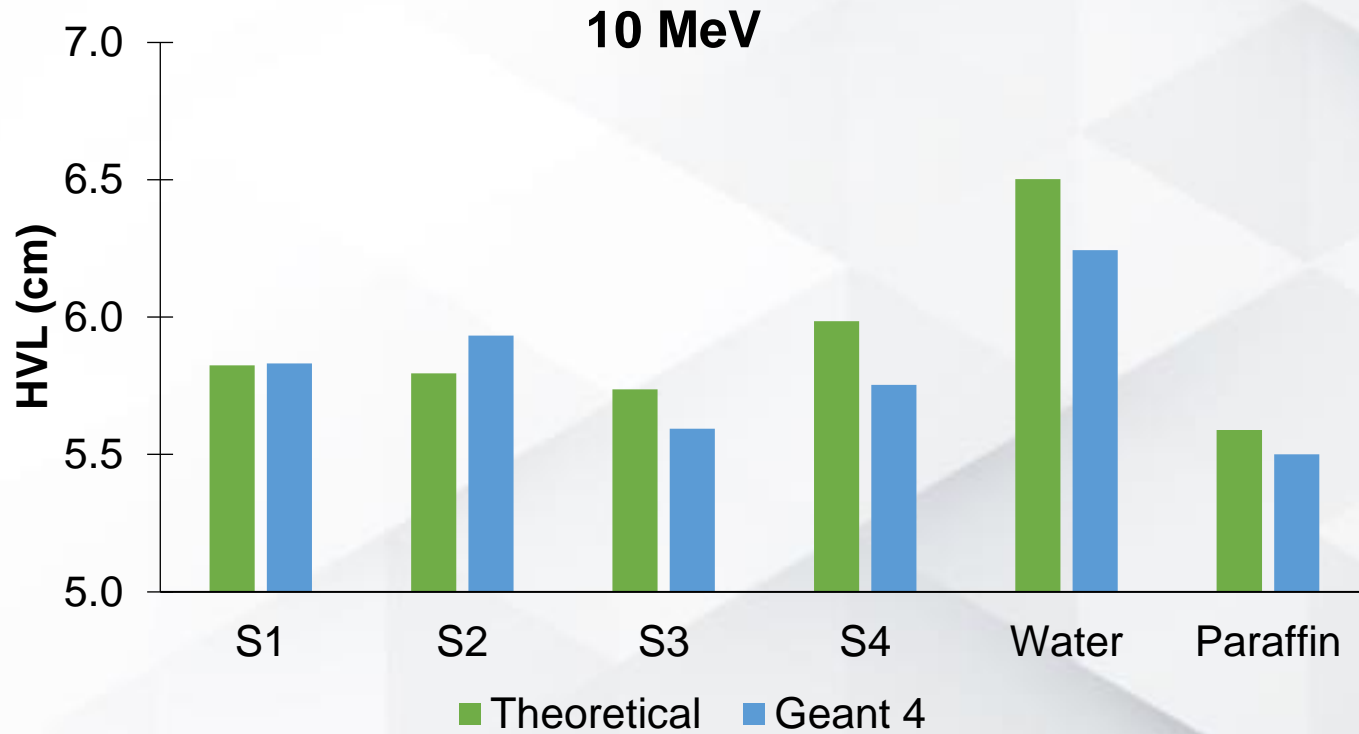
➤ Half Value Layers (cm) for cement-polymer composites:



➤ Half Value Layers (cm) for cement-polymer composites:



➤ Half Value Layers (cm) for cement-polymer composites:



5. Conclusion

- This study investigated the shielding properties of cement-based composite materials for thermal, epithermal, and fast neutrons.
- The theoretical calculations and Monte Carlo simulations were conducted.
- The thermal neutron shielding performance of the composite containing 4 wt.% cement B4C was better than that of all composites, water, and paraffin.
- The thermal neutron shielding of the composite containing 2 wt.%/cement B4C and 2 wt.%/cement PANI was better than that of water.
- Due to PANI's hydrogen content, samples containing PANI had slightly better neutron shielding performance than other samples at 1 keV.
- The samples' shielding performance against fast neutrons is superior to that of water. Paraffin has a slightly better shielding performance than the samples.
- Experimental studies are ongoing.

THANK YOU!



Acknowledgments

18

The Istanbul Technical University Scientific Research Projects Coordination Unit (Project No: 45205) supported this work.