

ISINN-30



Progress in neutron resonance imaging experiments using

MCP at CSNS Back-n white neutron facility

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- Background
- Experiment Setup
- Data Processing and Analysis
- Result



Neutron Radiography (NR), is a non-invasive method to characterize the internal structure of materials

Neutron Resonance Transmission Analysis (NRTA), is a method utilizing the different resonance peaks for the nuclide analysis.



Imaging of a old Buddha sculpture (photo left) with 150 kV X-ray (middle) and thermal neutrons (right)[1].



Pictures of the radioactive PbI_2 sample (photo upper) and its transmission spectrum(lower)[2].









Inorganic Scintillators [1].

Gated Cameras[2].



[1] Journal of Analytical Atomic Spectrometry, 2015, 30(3): 745-750.

[2] NIMA, 2023, 1048: 167892.

[3] Journal of Analytical Atomic Spectrometry, 2019, 34(12): 2420-2427.



Region A

0.8

uoissimsuis 0.6 0.4 0.2

0

5

Boron-doped microchannel plates (MCPs) balance the detection efficiency and spacial resolution



Schematic diagram of an MCP/Timepix event counting detector.

NRTI was used to study the nuclide composition of natural gold Samples based on boron-doped MCP. The images of Au and Pd are clear by selecting different energy ranges .

Pd only:

N3

500

Au only:

S0 Neutron energy (eV)

Region B

11

Boroned MCP [1][2].



The unique capability of NRTI makes research that can only be studied non-destructively very attractive, especially for valuable samples (e.g. rare geological samples, celestial objects or cultural heritages).



Photograph of imitated lunar soil sample.



Photographs of underwater relic found in South China Sea.



The NRTI experiment using boron-doped MCP at the back-streaming neutron line (Back-n) of the China Spallation Neutron Source (CSNS) has been conducted

CSNS Back-n

- The CSNS Back-n has a wide energy range(0.3 eV- 300 MeV), high neutron flux(1×10⁷@ES#2), and good energy resolution (0.32%- 4.9%)
- According to simulation, the counting rate of boron-doped MCP is high and does not vary significantly with neutron energy





NRTI based on MCP at CSNS Back-n can achieve neutron detection in the energy range of eV- tens of MeV, thereby covering the vast majority of nuclides



□ Sample



The details of the samples

Sample	Thikness(cm)	S(cm2)		101	(Bam)
С	0.1	0.5			
AI	0.1	sample support			
Fe	0.1	0.5	$10^{-1} 1 10 10^2 10^3 10^4 10^5 10^6 10^7 \\ E(Ram)$	$10^{-1} 1 10 10^2 10^3 10^4 10^5 10^6 10^7 10^{-1} 10^$	$\begin{bmatrix} & & & & & & \\ & & & & & & & \\ 10^{-1} & 1 & 10 & 10^2 & 10^3 & 10^4 & 10^5 & 10^6 & 10^7 \\ & & & & & & \\ E(\text{Rarn}) \\ & & & & \\ \hline \\$
Cu	0.1	0.5			0
Ag	0.01	1			
Au	0.01	1	10^{-1} 1 10 10 ² 10 ³ 10 ⁴ 10 ⁵ 10^{6} 10^{7} E_{n} (Barn)	$\underbrace{ \begin{array}{ccccccccccccccccccccccccccccccccccc$	10^{-1} 1 10 10^2 10^3 10^4 10^6 10^7 E_n (Barn)

Samples are cuboids / foils, except for the Al sample support

Samples used in the experiment and cross sections of (n,tot) reactions.

Au, Ag, Cu, Fe, Al, C samples was selected to cover the energy range from eV to MeV



D MCPs



Two kind of MCPs

Parameters	B-MCP	Ordinary MCP	
Diameter (mm)	56	56	
Thickness (µm)	480	480	
Pore inner diameter (µm)	8.8	8.8	
Pore area ratio (%)	~65	~65	
Gain	~115@800V	~10720@1000V	
¹⁰ B (mol%)	15	-	







Sample rate : **80 MSPS** Sampling resolution:**12 bit**

Analog-to-Digital Module

Schematic diagram MCP detection system.







Pre-Amplifier Module

Data Concentrator Module $_{10}$

Crossed Anonde Strip 128x128



D Experiment Condition



Photograph of the MCP Detector.

> Neutron flux: 1.025×10^6 n/cm²/s

- > Neutron beam spot: φ 30 mm
- ≻Experiment position: ES#2 L=~77m
- > Experiment duration: 2024 2.3-2.5 && 2024 3.4-3.7 (7 days)
- Bias voltage: B-MCP 800 V / ordinary MCP 800 900 V



Data Processing and Analysis

D Program Framework

The framework of data processing and analysis program was based on **BLUET-v4[1]**(A framework is developed for the data processing and analysis of advanced detectors)





□ Waveform analysis and event reconstruction



Signal selection using fitting parameters.

Event hit after event reconstruction.

Original waveform(a) and waveform after baseline subtraction with the red line indicating the fitting result (b).



Data Processing and Analysis



6.5 lg(TOF) (ns)

hlogtoflogamp_px

6.5 lg(TOF) (ns)

Entries

2.220656e+07

4.321

1.323



Data Processing and Analysis

D Open Beam Data



Reaction rate as a function of neutron energy.

The **resonant structure** due to spallation target can be observed clearly



The spatial profile of the neutron beam. (0.25 mm/bin)

The **spacial resolution** is good by using the charge center-of-mass method











Neutron transmission images in resonance peaks range and neutron transmission spectra measured for different samples.

- The images were obtained by normalization of the "sample in" data by the "open beam" data measured separately.
- The experimental transmission was obtained in the sample area.







Neutron transmission images in different energy range and neutron transmission spectra measured for different samples.





- The online NRTI experiment using boron-doped MCP at CSNS Back-n was conducted
- The main functions of the data processing and analysis program have been implemented
- NRTI of simple samples (Au, Ag, Cu, Fe, Al) has been achieved





Thank you





Pile-up signal



Comparation of single bunch and double bunch mode